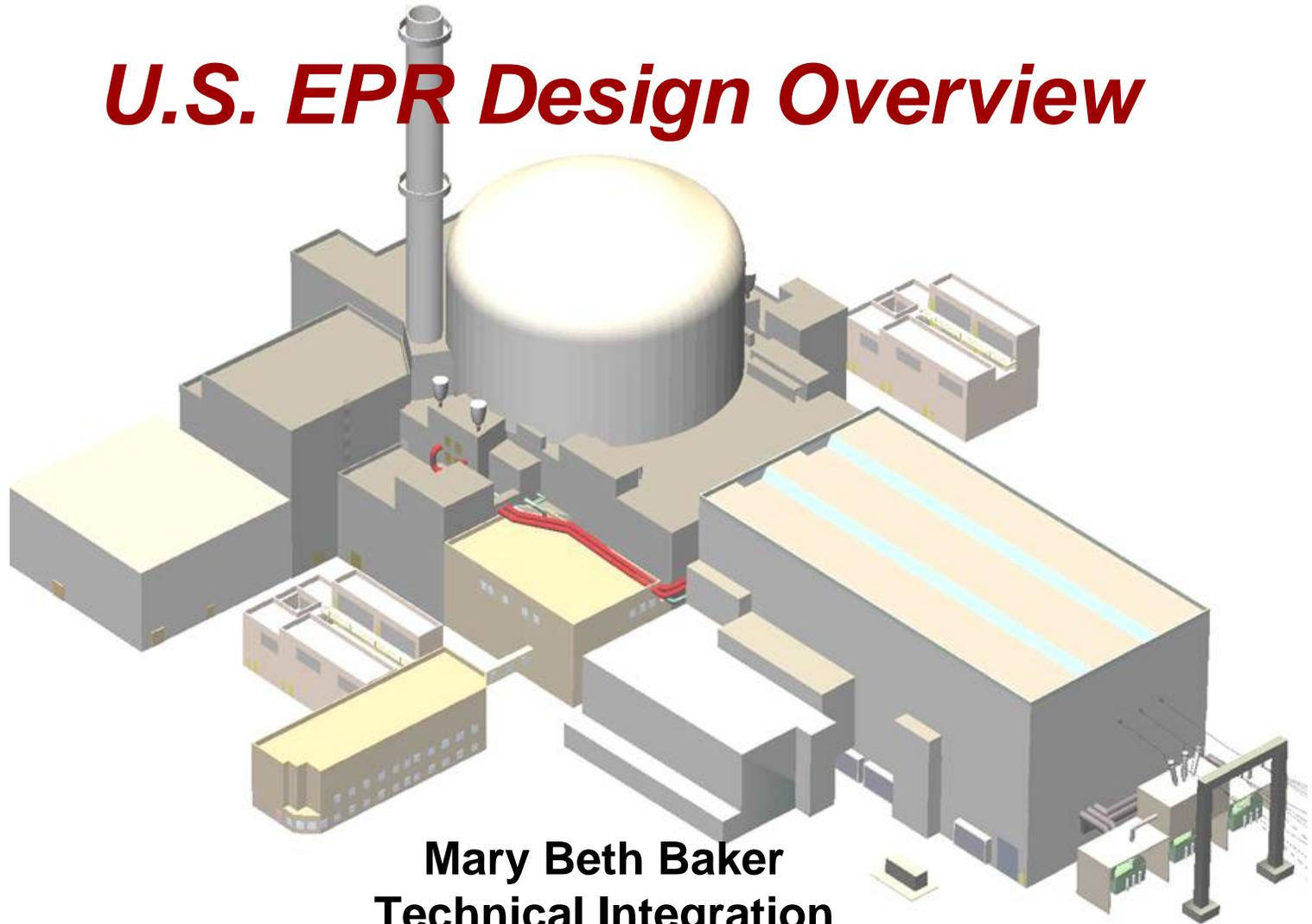


# ***U.S. EPR Design Overview***

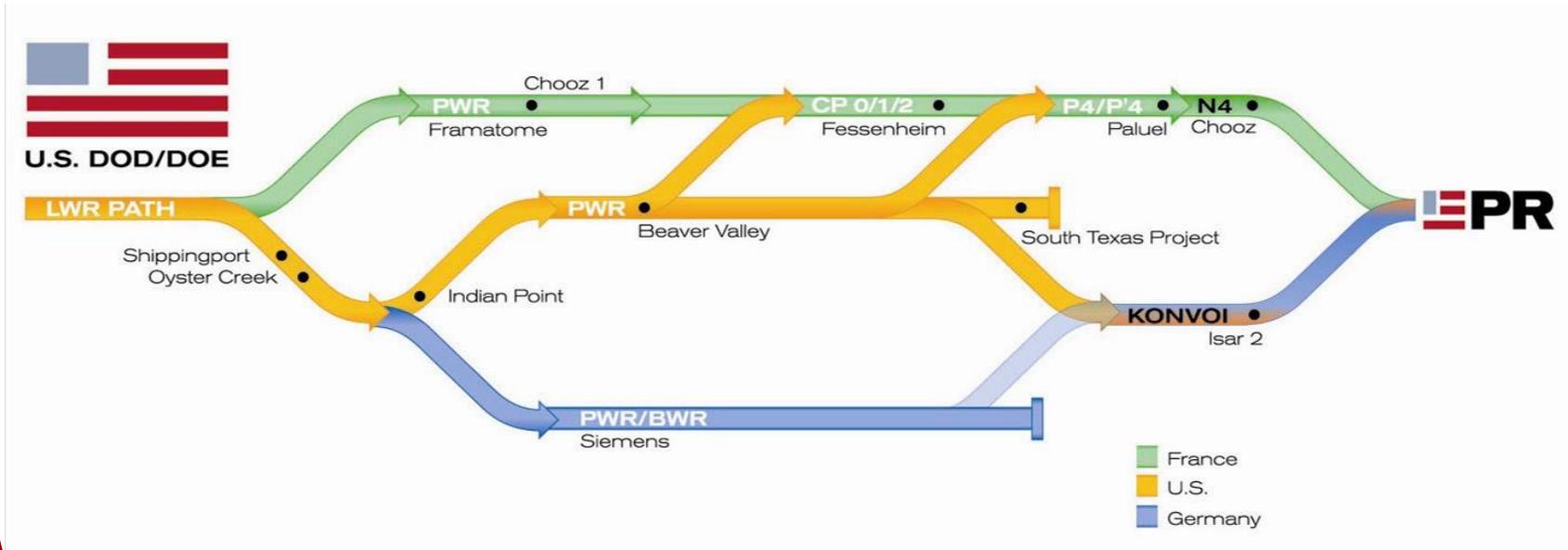


**Mary Beth Baker  
Technical Integration  
AREVA NP, Inc.**



# Design Heritage

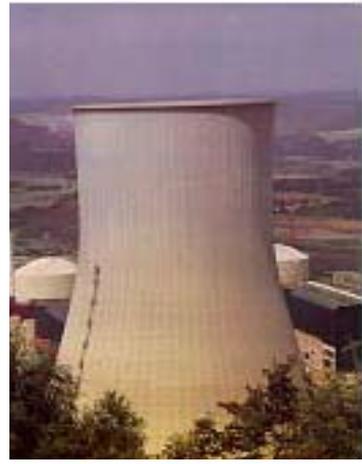
- EPR is a global product based on U.S. technology and experience that have been advanced to the next level.



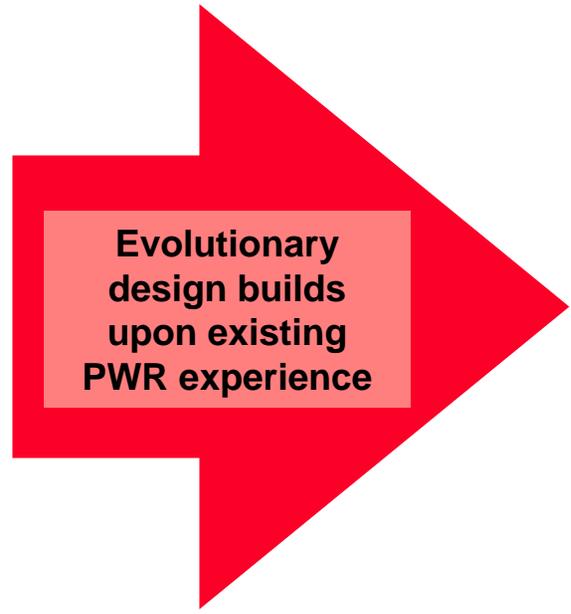
***A mature design based on familiar technology***

# *Experience from 77 operating PWRs in France & Germany*

**N4**



**KONVOI**

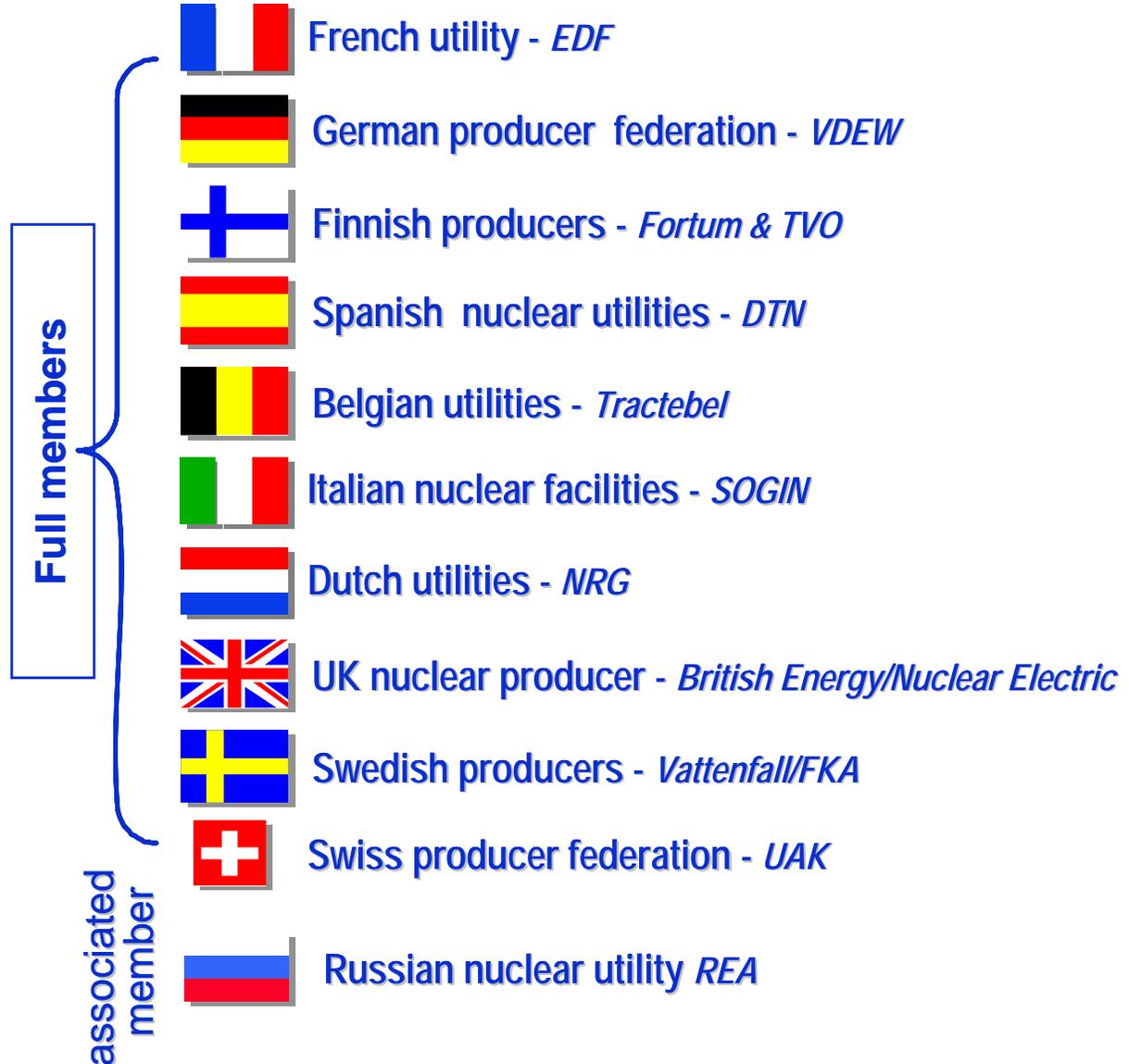


**EPR**

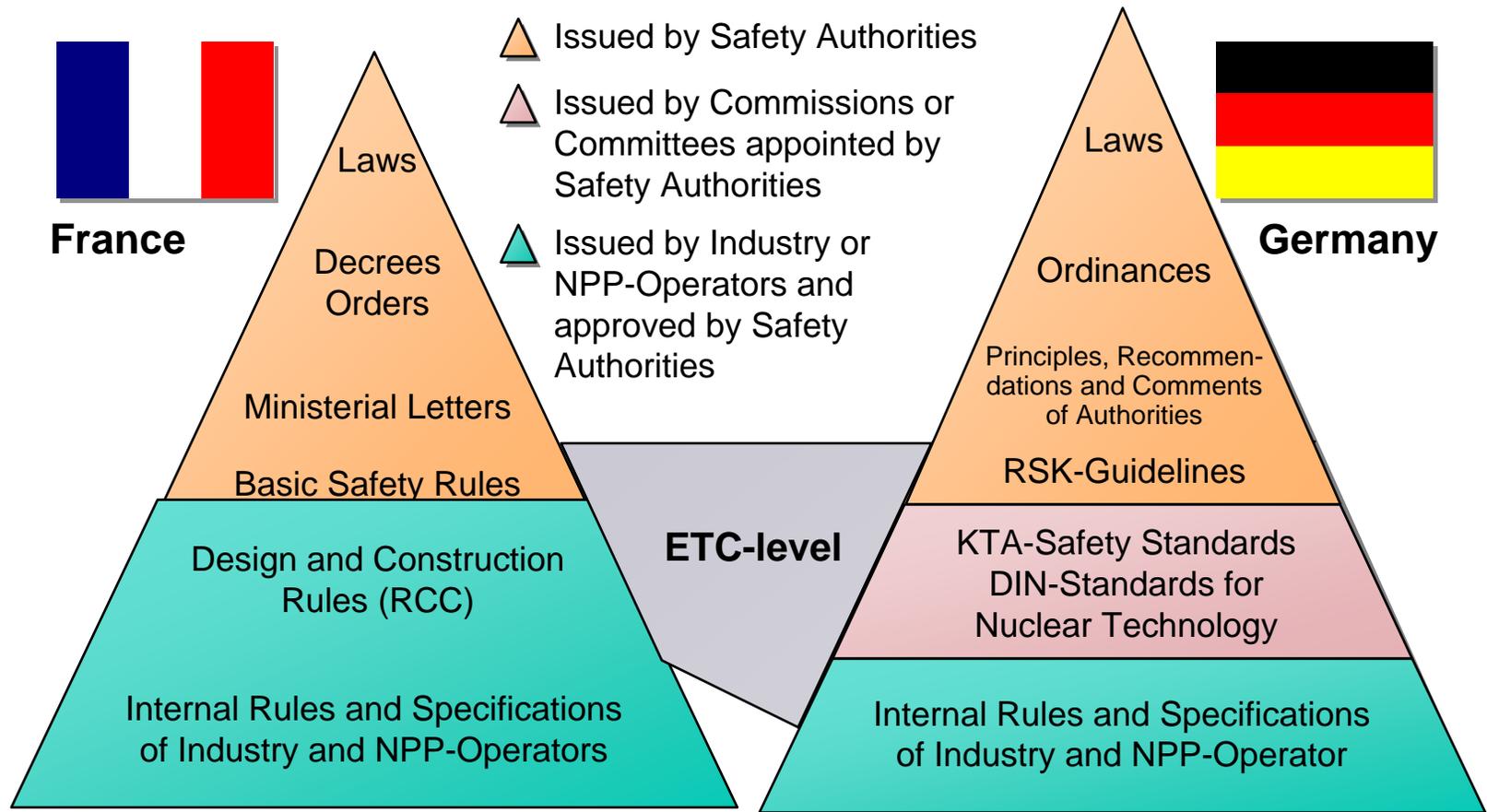


***Solid Basis of Experience with Outstanding Performance***

# European Utility Participants

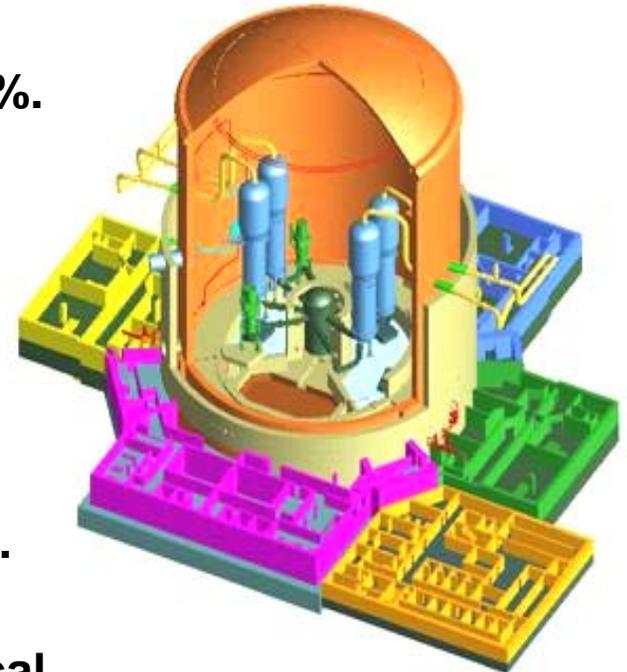


# The Harmonization of Codes and Standards



# ***EPR Development Objectives***

- **Evolutionary** design based on existing PWR construction experience, R&D, operating experience and “lessons learned”.
- **Improved economics**
  - Reduce generation cost by at least 10%.
  - Simplify operations and maintenance.
- **Safer**
  - Reduce occupational exposure and LLW.
  - Increase design margins.
  - Reduce core damage frequency (CDF).
  - Accommodate severe accidents and external hazards with no long-term local population effect



# Major Design Features

## ➤ Nuclear Island

- *Proven Four-Loop RCS Design*
- *Four-Train Safety Systems*
- *Double Containment*
- *In-Containment Borated Water Storage*
- *Severe Accident Mitigation*
- *Separate Safety Buildings*
- *Advanced 'Cockpit' Control Room*

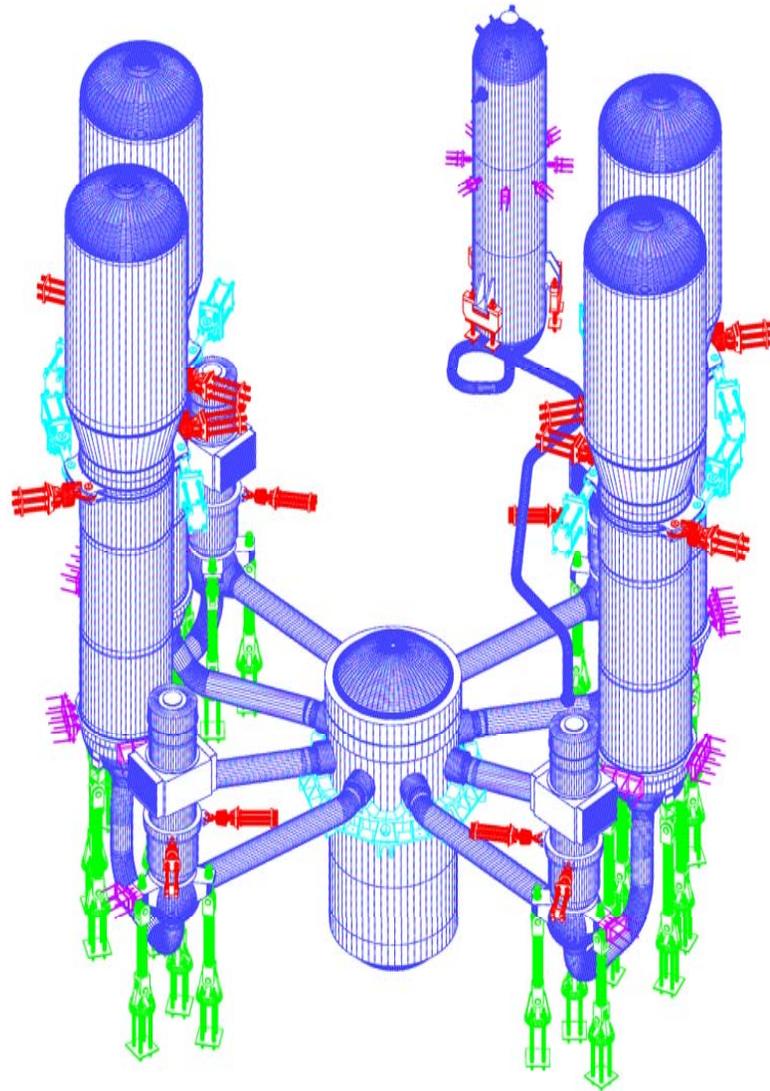
## ➤ Electrical

- *Shed Power to House Load*
- *Four Emergency D/Gs*
- *Two Smaller, Diverse SBO D/Gs*

## ➤ Site Characteristics

- *Airplane Crash Protection (military and commercial)*
- *Explosion Pressure Wave*

***Reflects full benefit of operating experience and 21<sup>st</sup> century requirements.***



- Conventional 4-loop PWR design, proven by decades of design, licensing & operating experience.
- NSSS component volumes increased compared to existing PWRs, increasing operator grace period for many transients and accidents

***A solid foundation of operating experience.***

# Key Plant Parameters

| Parameter                                 | Typical 4-Loop (Uprated) | U.S. EPR |
|---|--------------------------|----------|
| Design Life                               | 40                       | 60       |
| Thermal Power, MW                         | 3587                     | 4590     |
| Electrical Power (Net), MW                | 1220                     | 1600     |
| Plant Efficiency, Percent                 | 34                       | 35       |
| Hot Leg Temperature, F                    | 619                      | 624      |
| Cold Leg Temperature, F                   | 559                      | 563      |
| Reactor Coolant Flow Per Loop, gpm        | 100,500                  | 125,000  |
| Primary System Operating Pressure, psia   | 2250                     | 2250     |
| Steam Pressure, psia                      | 1000                     | 1109     |
| Steam Flow Per Loop, Mlb/hr               | 4.1                      | 5.17     |
| Total RCS Volume, cu.ft.                  | 12,265                   | 16,245   |
| Pressurizer Volume, cu.ft.                | 1800                     | 2649     |
| SG Secondary Inventory at Full Power, lbm | 101,000                  | 182,000  |

***Increased power and thermal efficiency***

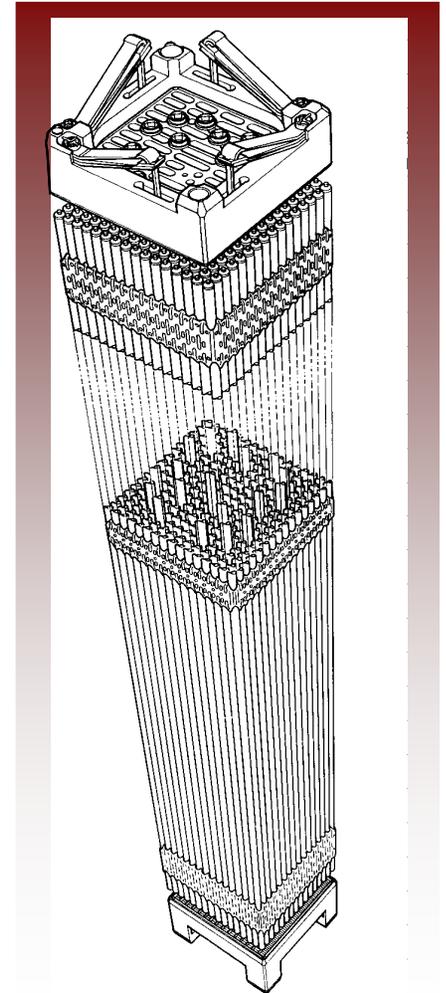
# Core Characteristics

- Increased Uranium utilization (~7% reduction in uranium consumption)
- Designed for use of MOX fuel
- Designed for 12 to 24 month fuel cycle
- Up to 5% enrichment
- > 60 GWd/t burn-up

***Designed for increased flexibility & performance***

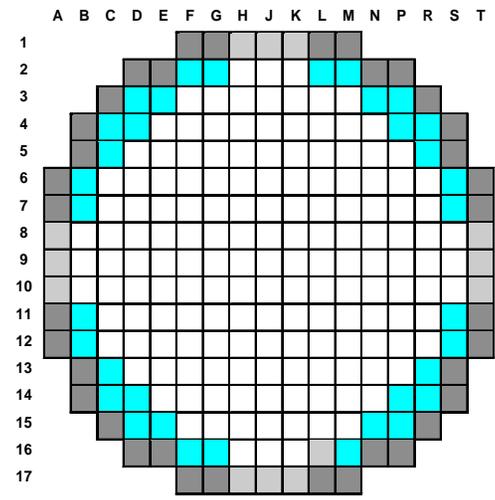
# Fuel Design Proven By Operation

- 17x17
- Typical Pitch-to-Diameter Ratio
- M5 Cladding
- Heated Length Similar to N4
- M5 HTP Mixing Vane Grids
- Anti-Debris Lower End Fitting
- Significant Design Margins
- MOX Compatible



# EPR Core Design Parameters

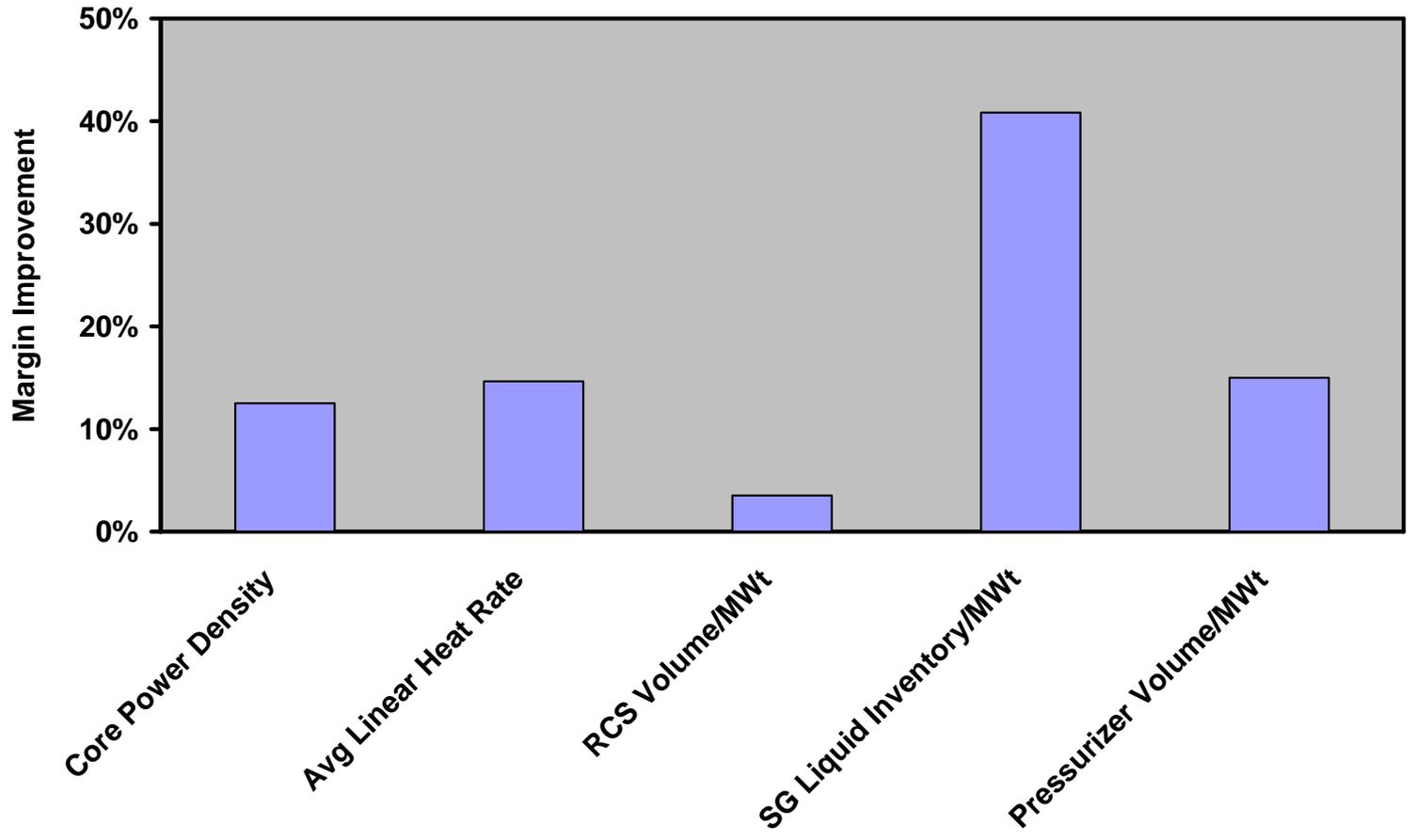
| Parameter                       | Current 4-Loop (Uprated) | EPR   |
|---------------------------------|--------------------------|-------|
| Core Thermal Power, MW          | 3587                     | 4590  |
| Number of Fuel Assemblies       | 193                      | 241   |
| Fuel Lattice                    | 17x17                    | 17x17 |
| Active Fuel Length, ft          | 12                       | 13.78 |
| Rods Per Assembly               | 264                      | 265   |
| Average Linear Heat Rate, kw/ft | 5.84                     | 5.10  |
| Peak Linear Heat Rate, kW/ft    | 14.6                     | 14.0  |
| Number of Control Rods          | 53                       | 89    |



| Type of Plant   | No of Fuel Assy |  |
|-----------------|-----------------|--|
| 4-loop 1300 MWe | 193             |  |
| 4-loop N4       | 205             |  |
| U.S. EPR        | 241             |  |

# Improved Design Margin

Margin Comparison of EPR to Current 4-Loop Plant

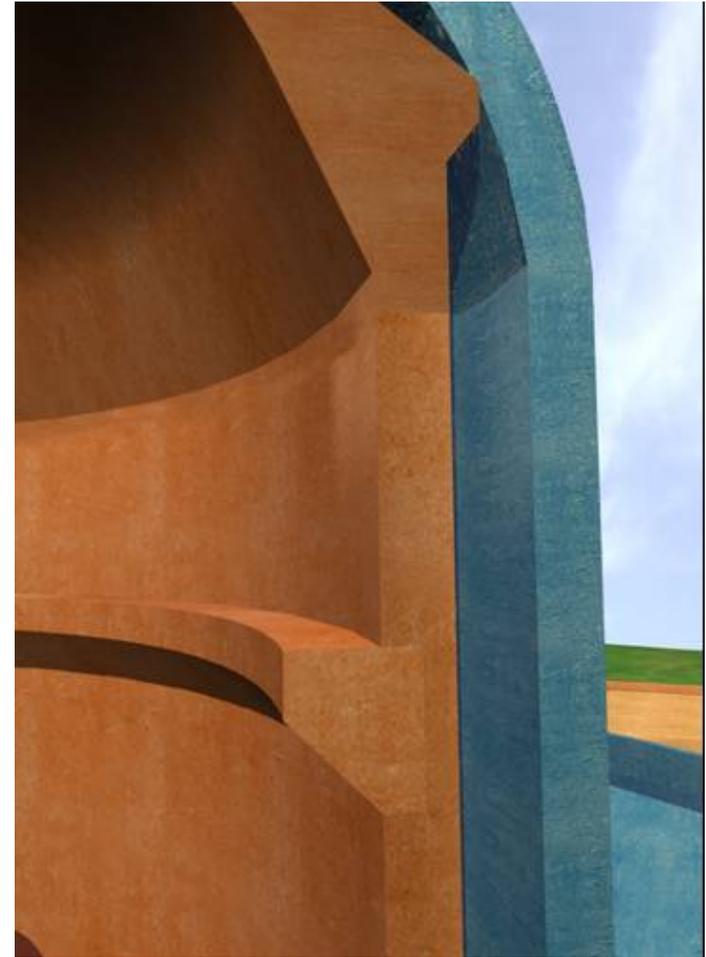


***Increased power with improved margins.***

# *Selected Key Features of EPR*

# Double-Walled Containment

- Inner wall post-tensioned concrete with steel liner
- Outer wall reinforced concrete
- Protection against airplane crash
- Protection against external explosions
- Annulus sub-atmospheric and filtered to reduce radioisotope release

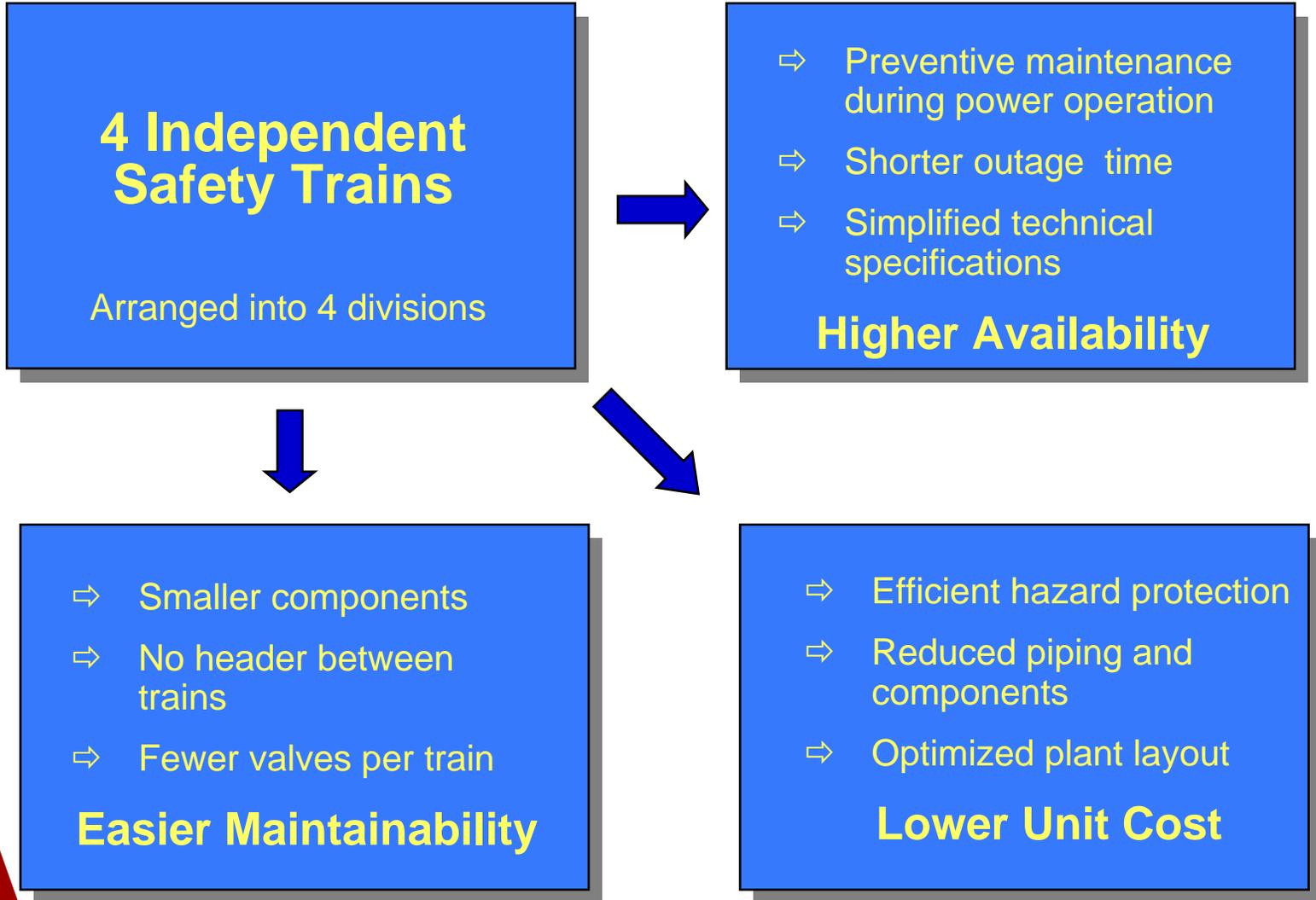


# *The Four Train Concept*

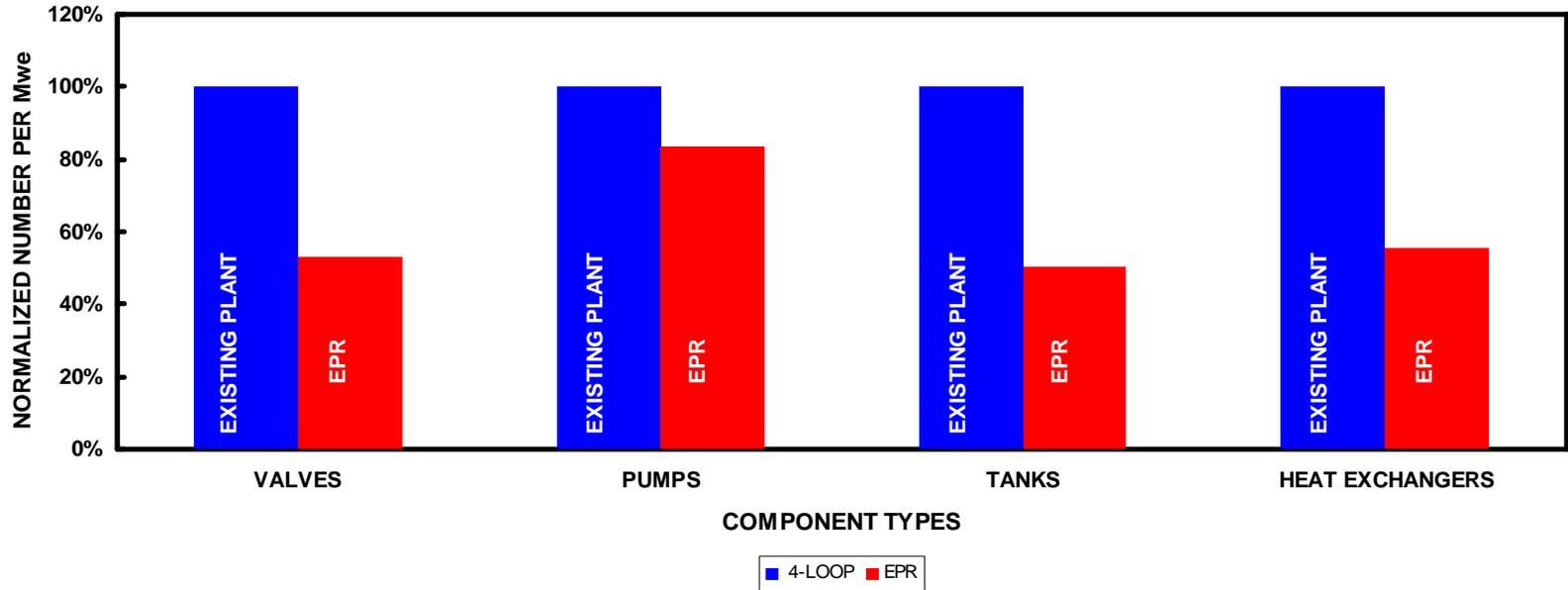


***Each safety train is independent and located within a physically separate building.***

# The Four Train Concept (cont'd)



# Reduced Equipment Quantities

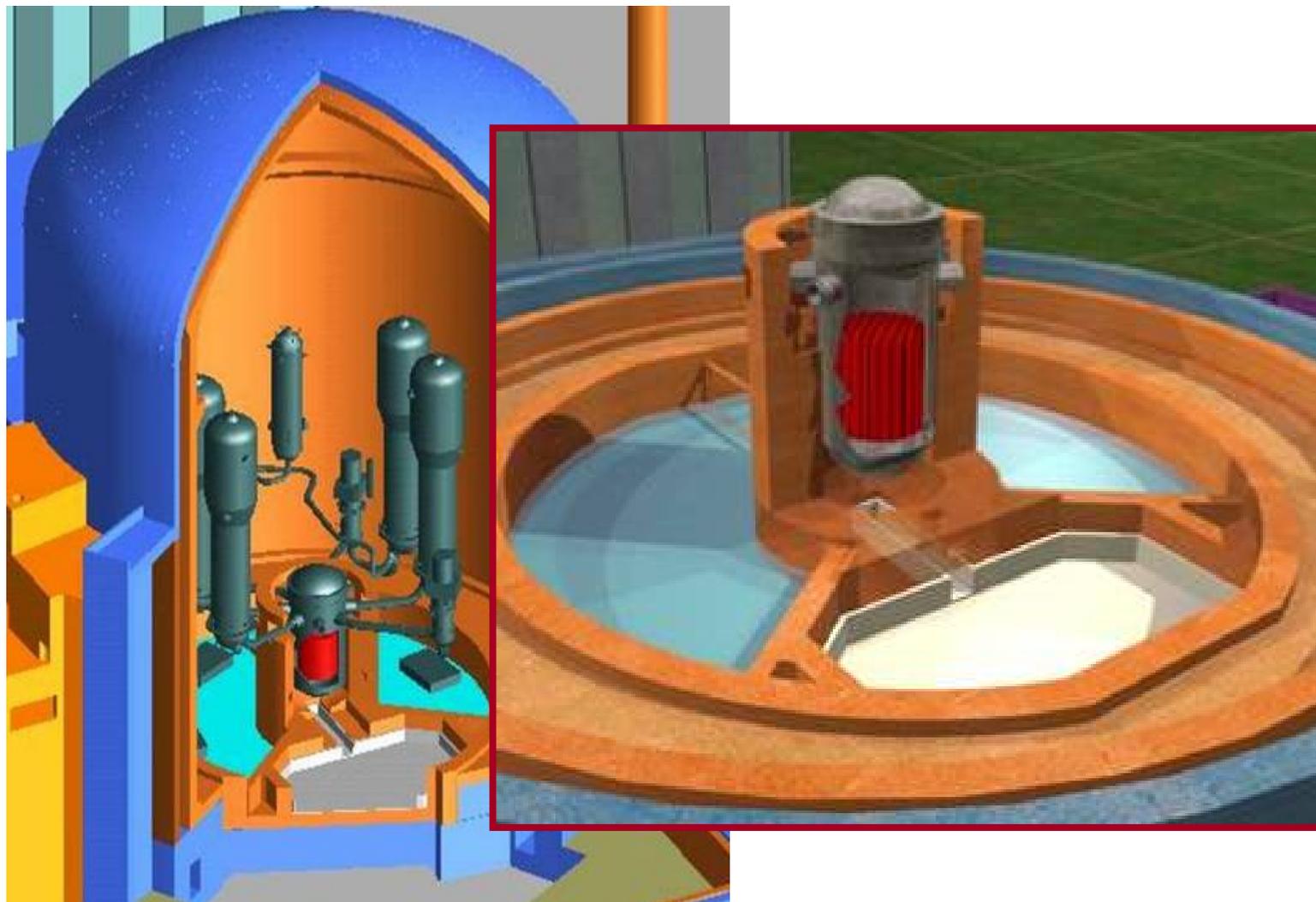


Study based on: RCS, Pzr. Spray, RCP seal and Leakoff, SI/RHR, CVCS incl. Boration and Demin/Seal Water, SFP Cooling, CCW, FW AFW/EFW, and MS

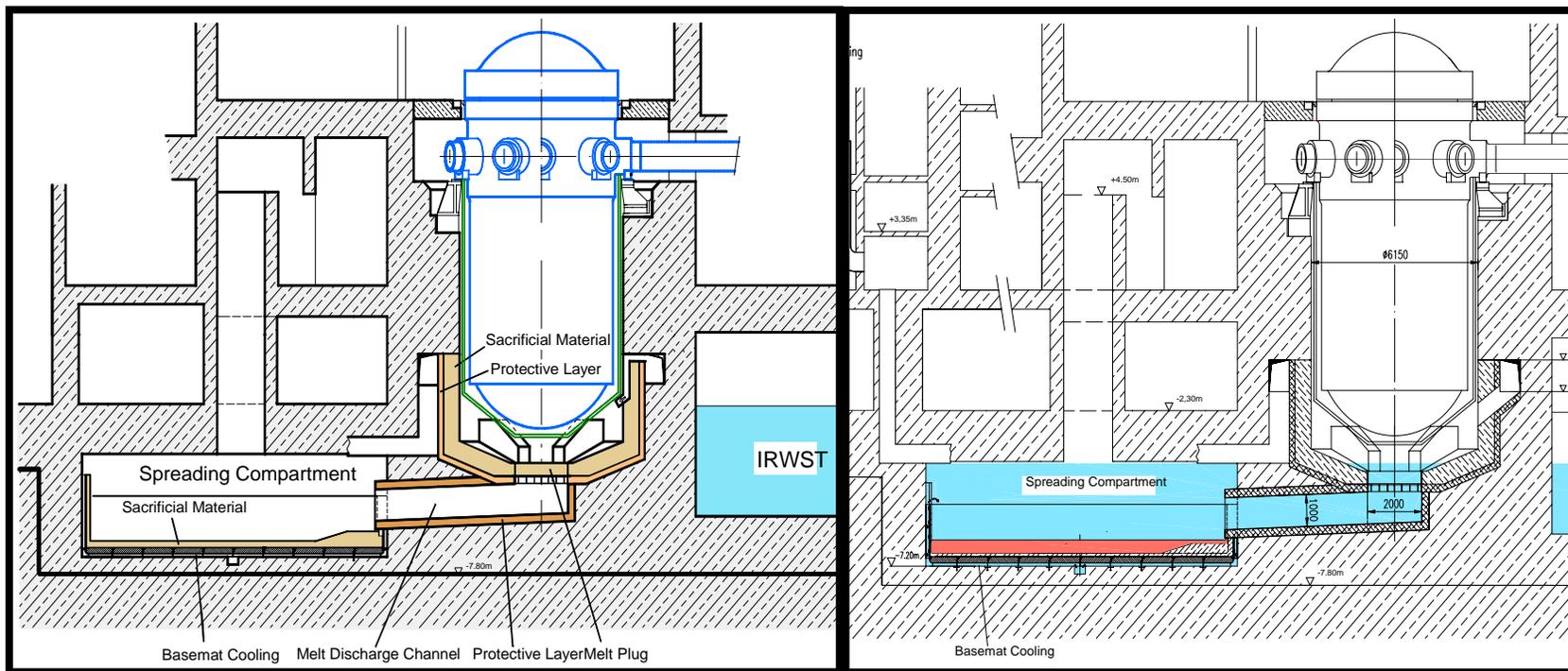
|                             | EPR   | 4-Loop PWR | % Change (Absolute) | % Change (Count/MWe) |
|-----------------------------|-------|------------|---------------------|----------------------|
| <b>Pumps &amp; Turbines</b> | 43    | 37         | 16                  | (16)                 |
| <b>Heat Exchangers</b>      | 34    | 44         | (23)                | (44)                 |
| <b>Tanks</b>                | 23    | 33         | (30)                | (50)                 |
| <b>Valves</b>               | 2,044 | 2,766      | (26)                | (47)                 |

\* Information based on AREVA study of Modern 4-Loop facility

# Severe Accident Mitigation: Views of Corium Spreading Area & IRWST



# Severe Accident Mitigation: IRWST Provides Passive Cooling of Corium



# *Operator-Friendly Man-Machine Interface*



N4 Control Room



EPR Control Room

***Capitalizing on nuclear digital I&C operating experience and feedback.***

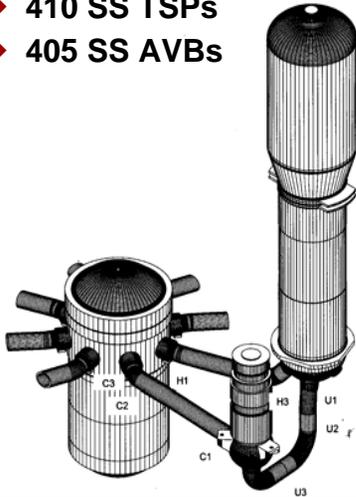


# Equipment Improvements

**Extensive use of forgings with integral nozzles.**

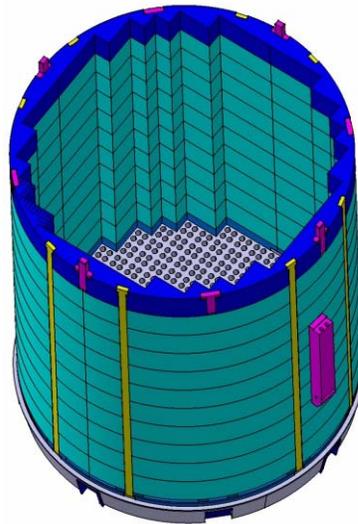
**Materials resistant to corrosion and cracking**

- ◆ 304L SS hot/cold legs
- ◆ 316L SS surge line
- ◆ 304L/316L RV internals
- ◆ 308/309 SS cladding
- ◆ Alloy 690 SG tubes
- ◆ 410 SS TSPs
- ◆ 405 SS AVBs

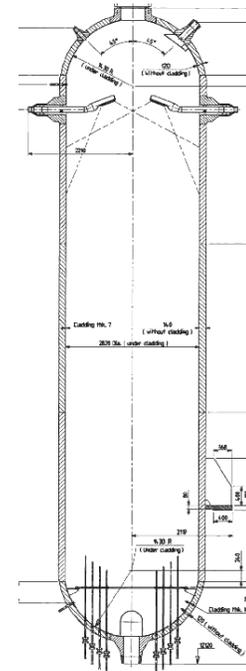


**Conventional core baffle replaced by heavy reflector.**

- ◆ Eliminates bolting
- ◆ Improves neutron economy
- ◆ Reduces vessel fluence

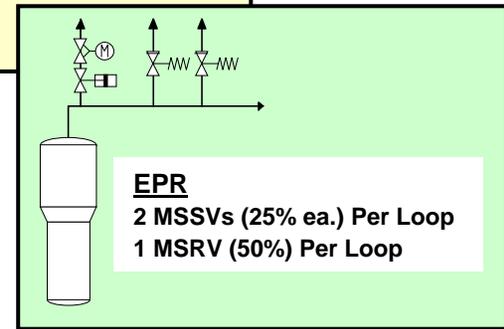
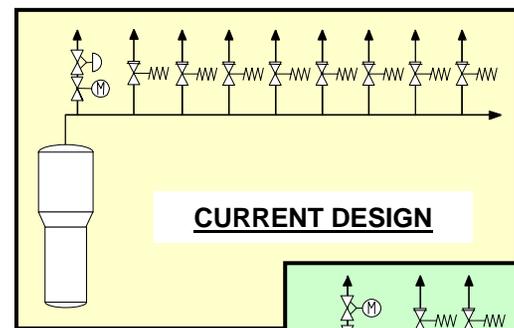
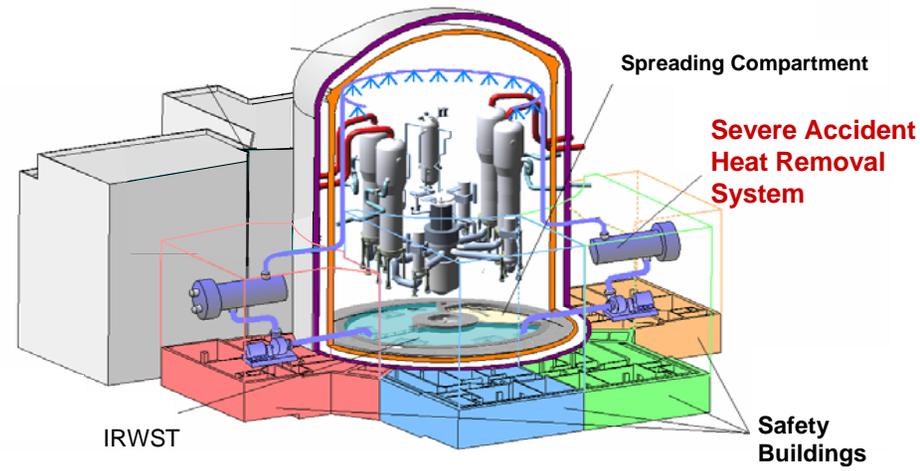


**Two normal pwr spray (ea. from different CL) plus one aux spray**



# Reduced Maintenance & Surveillance Testing

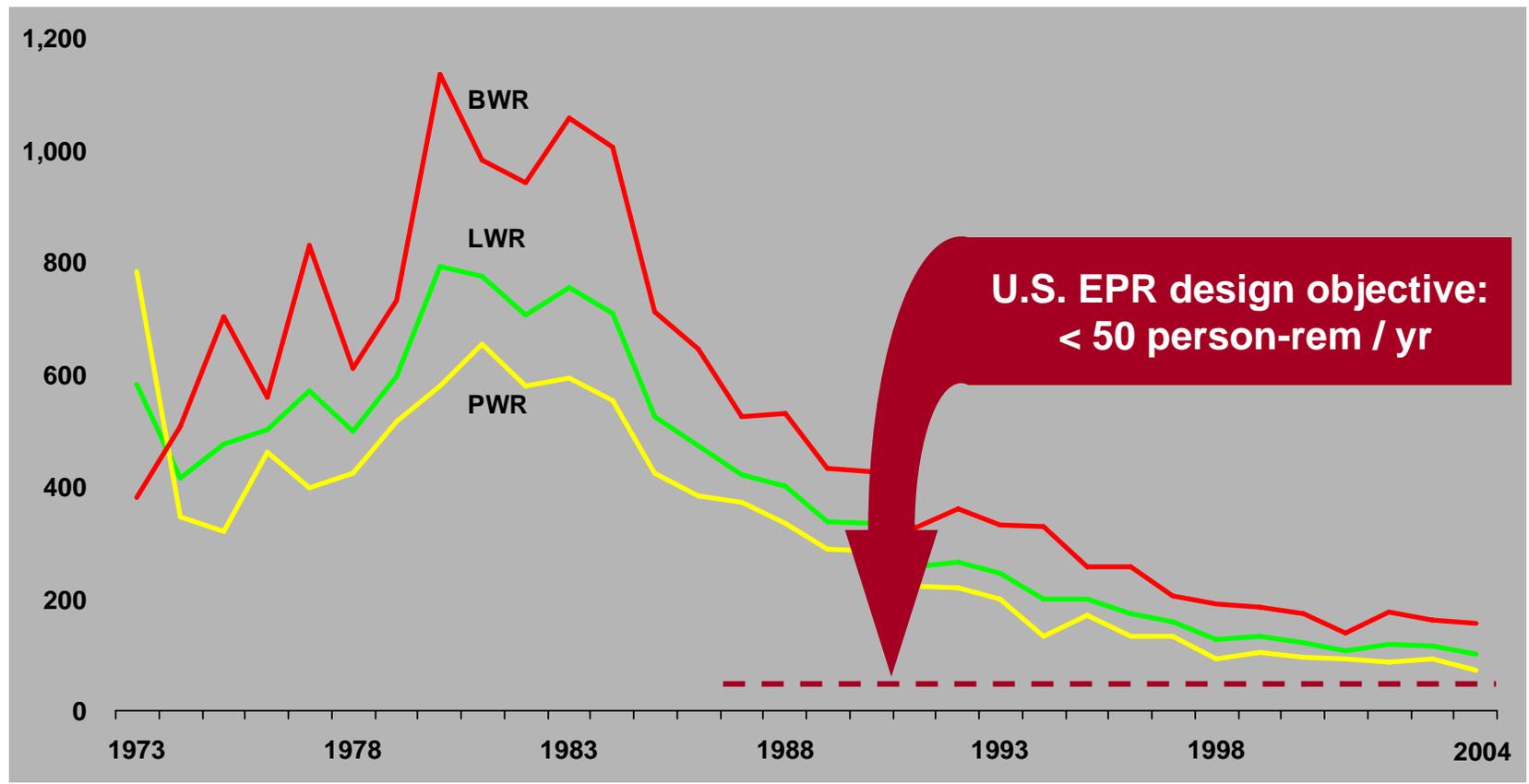
- No containment fan coolers
- Containment spray is non-safety (for severe accident)
- No turbine-driven MFW or AFW pumps (all electric)
- Main steam relief & safety valves reduced from 8 - 10 to 3 per loop



# ***Additional OE Feedback***

- **Elimination of Single-Point Vulnerabilities**
  - ◆ **Four, 33% Main Feedwater Pumps**
  - ◆ **Three, 50% Condensate Pumps**
  - ◆ **By-pass of components for maintenance w/no derate**
  - ◆ **Duplicates of key components (e.g., demins, Hx's) to allow isolation for maintenance**
  
- **Layout to Facilitate Maintenance**
  - ◆ **Room for access designed in**
  - ◆ **Most components can be removed and replaced via pre-designed pathways and equipment hatches**
  
- **ALARA**
  - ◆ **Minimize Cobalt in plant components**
  - ◆ **Vessels and Hx's designed to minimize deposits**
  - ◆ **Use of "Hot" and "Cold" zones**

# U.S. Industry-Average Dose Per Reactor 1973-2004, (Person-rem)

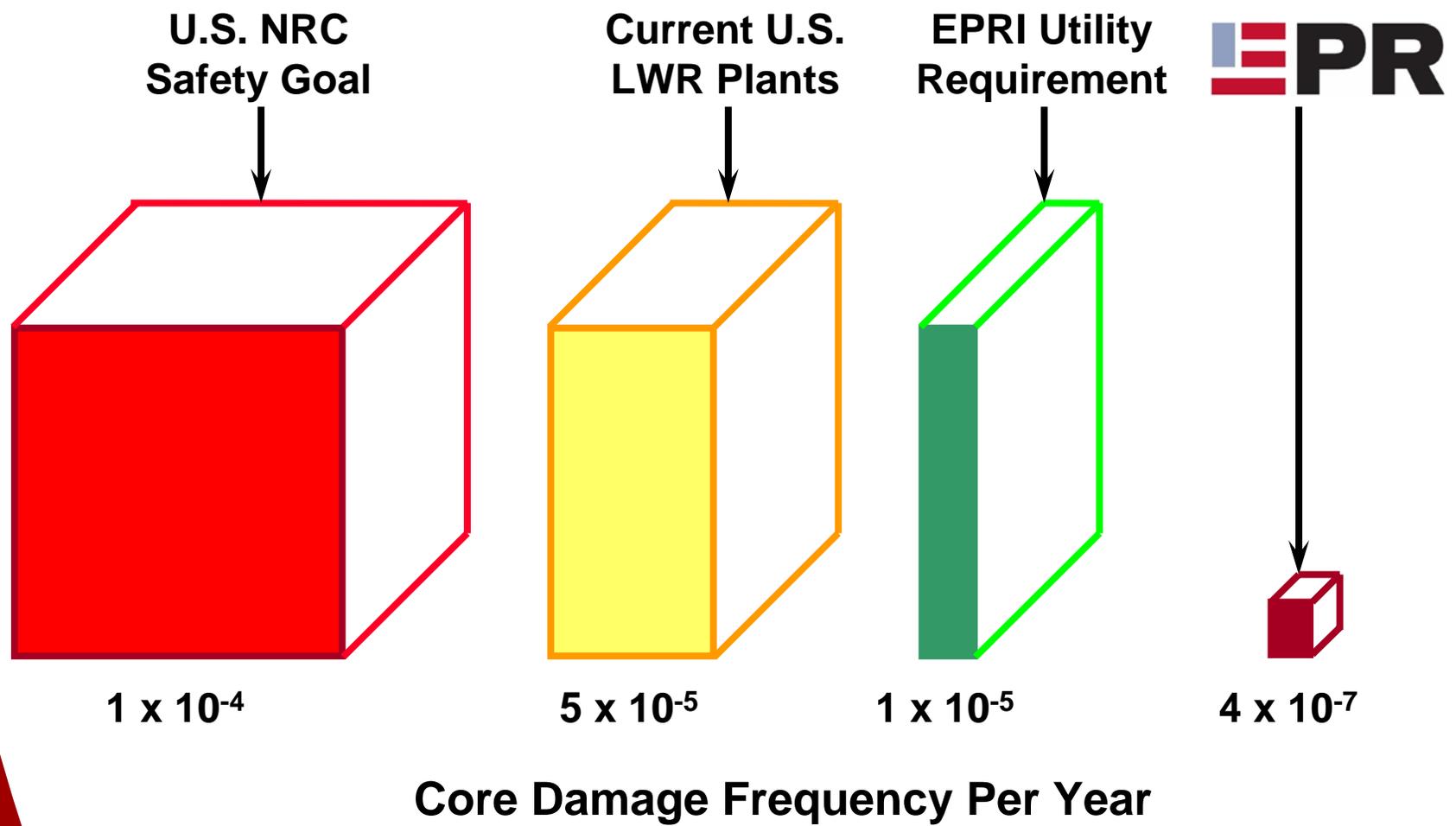


Source: Nuclear Regulatory Commission Occupational Radiation Exposure at Commercial Nuclear Power Reactors and Other Facilities 2004  
Updated: 4/06

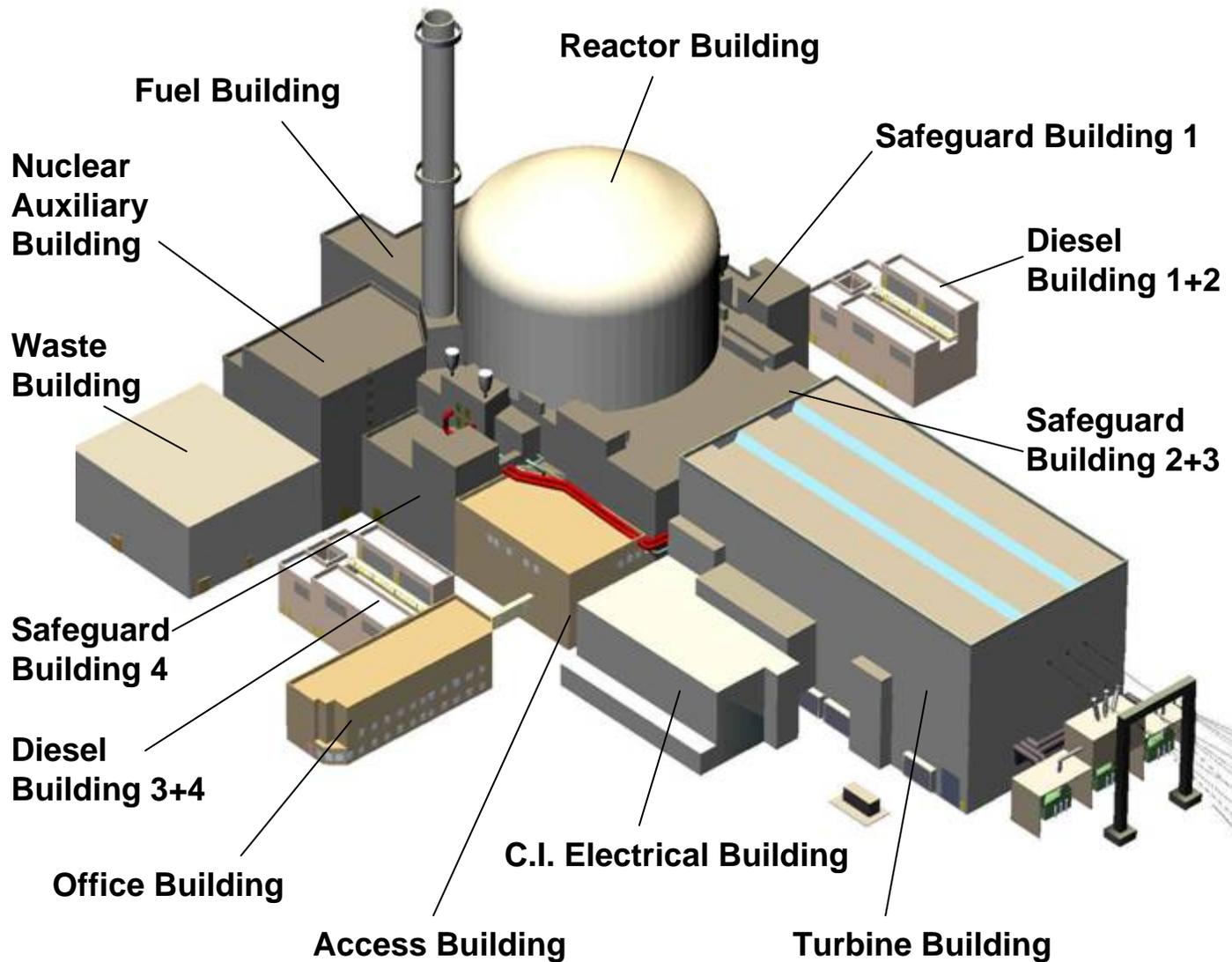
# ***Probabilistic Objectives And Targets***

- **Safety objective for integral core melt frequency (all plant states, all types of initiators):  $< 10^{-5}$  per year**
  
- **Design target for core melt frequency for internal events**
  - ◆ **from power states:  $< 10^{-6}$  per year**
  - ◆ **from shutdown states: less than power states**
  
- **Design target for core melt with large and early releases from containment:  $< 10^{-7}$  /year**

# U.S. Nuclear Industry Safety Goals

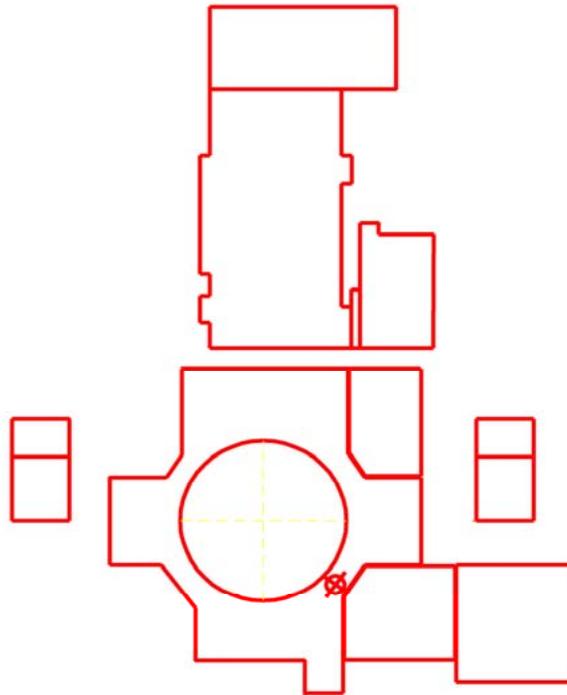


# General Plant Layout

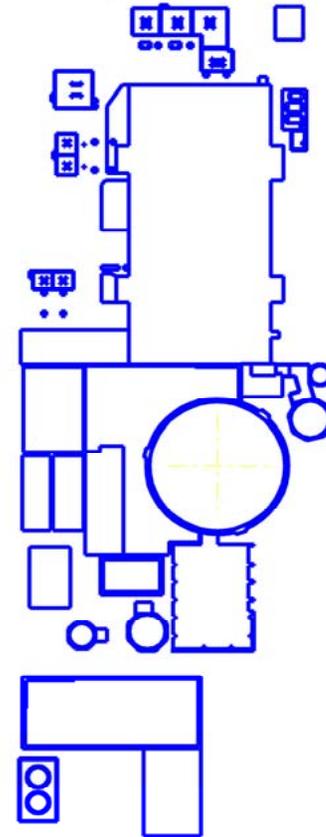


# U.S. EPR vs. Current Unit

**U.S. EPR  
1600 MWe**



**4-Loop Unit  
1235 MWe**



- **EPR is evolutionary**
- **Most features are typical of operating PWRs**
- **Features included to**
  - ◆ **Improve Safety**
    - Increase redundancy & separation
    - Reduce core damage frequency
    - Reduce large early release frequency
    - Mitigate severe accident scenarios
  - ◆ **Protect critical systems from external events**
    - Aircraft Hazard
    - External Explosion
    - Flood
  - ◆ **Improve human factors**
  - ◆ **Lower O&M Costs**
    - Simplified Systems
    - On-Line Maintenance
    - Use of latest, proven technology
    - Economy of Scale